

# H-Sensors and Fusion Work at SNL-CA

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Thomas Lemp and Thomas Friedman (SNL-NM)



Useful discussions with Peter Stangeby (Univ. of Toronto)  
And Bob Bastasz (SNL-CA retired)

## Outline

Overview of Hydrogen Work  
Details of Fusion Program on PSI  
Motivation for Atomic Hydrogen Measurements  
Pd-MOS Hydrogen Sensors

Tritium Focus Group Meeting, INL, September 23-25, 2014



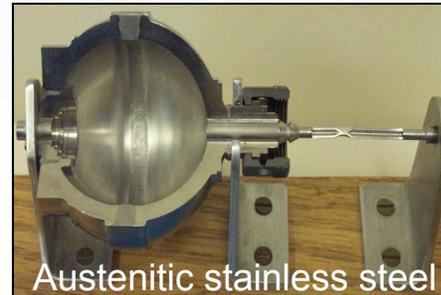
Sandia National Laboratories is a multi program laboratory managed and operated by Sandia Corporation, a wholly owned subsidiary of Lockheed Martin Corporation, for the U.S. Department of Energy's National Nuclear Security Administration under contract DE-AC04-94AL85000.



# Building a science-based understanding of hydrogen (and helium) behavior in materials

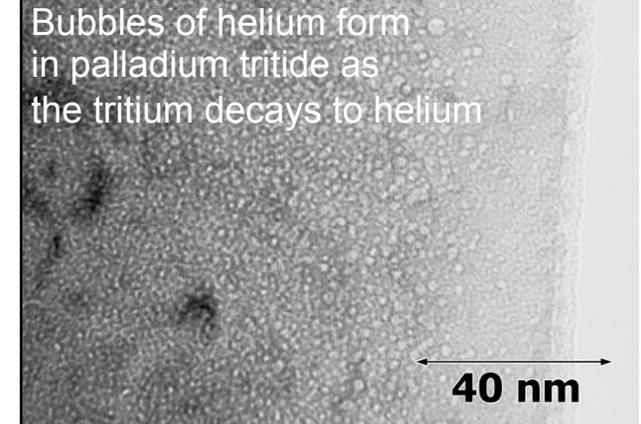
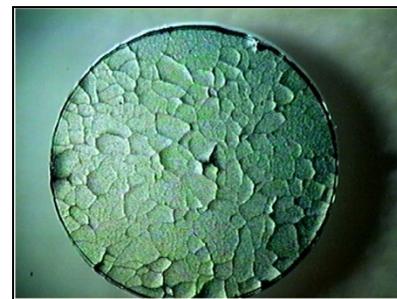
- **High pressure hydrogen storage for Gas Transfer Systems and hydrogen / fuel cell industry** →

- Embrittlement, permeation, trapping, microstructural effects, corrosion, Codes and Standards (H<sup>2</sup> safety)



- **Metal hydride studies for hydrogen storage** →

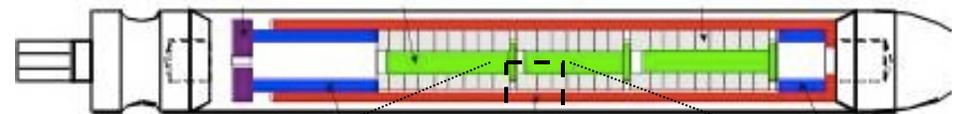
- He trapping from T decay, film adhesion, aging, new materials



- **Tritium production (TPBAR)**

- T migration in a complex materials environment

## Tritium Producing Burnable Absorber Rods

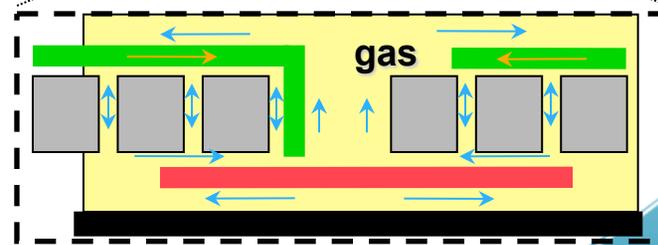


Zircoloy liner

LiAlO<sub>2</sub> pellet

NPZ getter

Cladding



- **Plasma-surface interactions (PSI) studies needed for magnetic fusion energy**

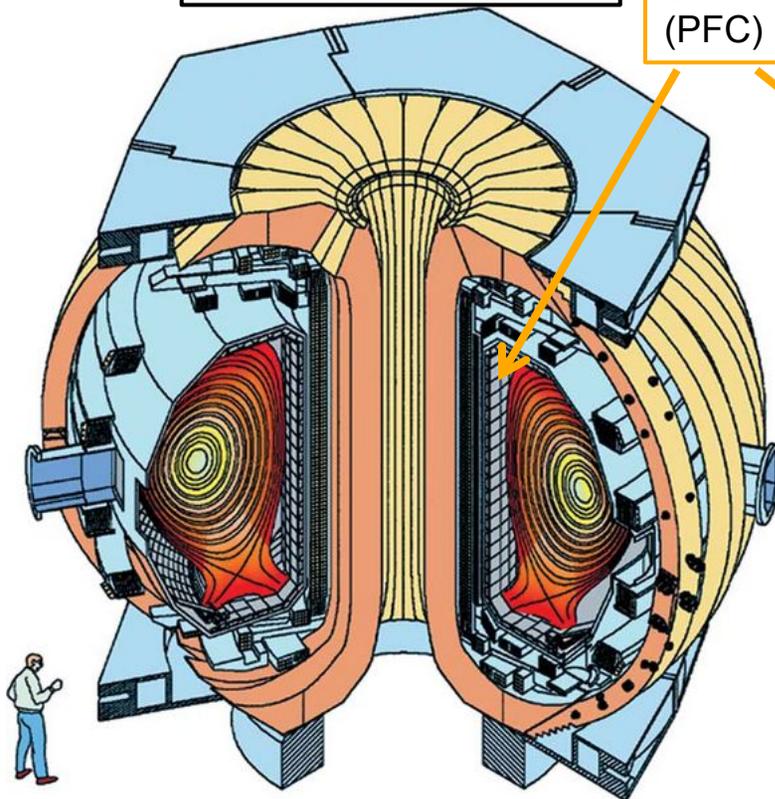
R&D efforts span a number of applied programs at SNL-CA

# Magnetic fusion energy research

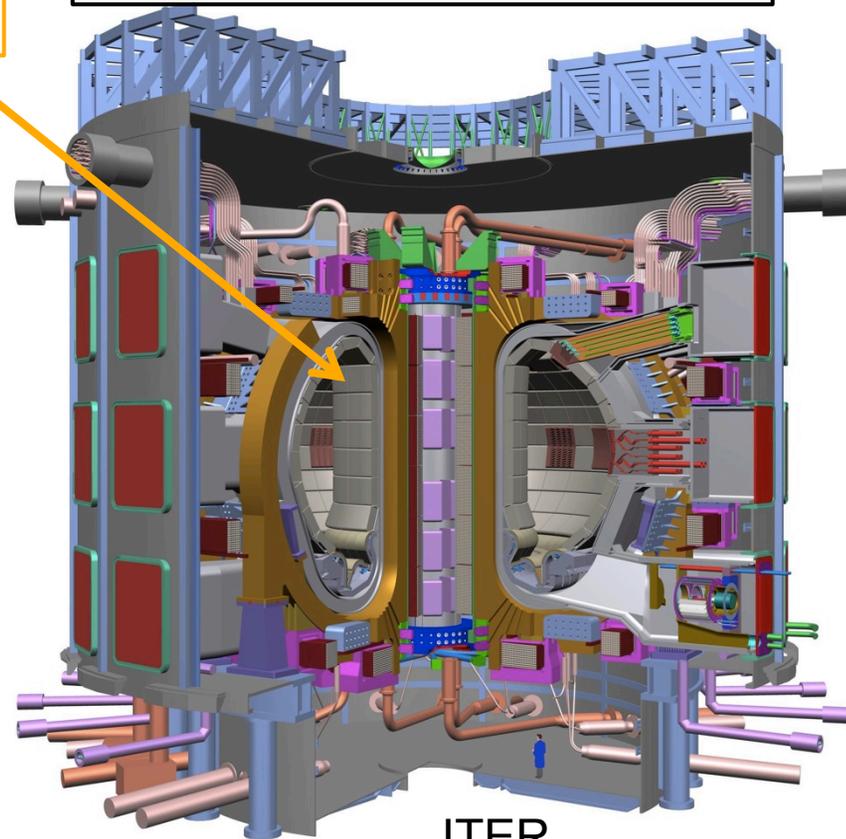
US research focused on the tokamak confinement scheme

Plasma Facing Surface (PFC)

Power and particle loading of the plasma facing components will limit fusion performance



DIII-D Tokamak at General Atomics

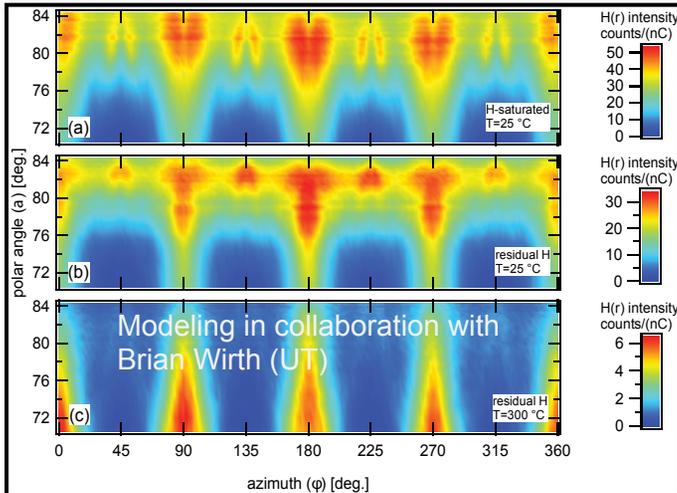


ITER

Greenwald report, ReNeW study, and OFES all agree on importance of PSI/PFC

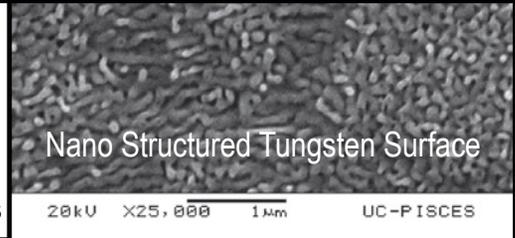
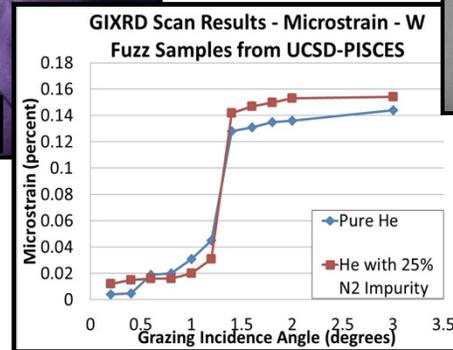
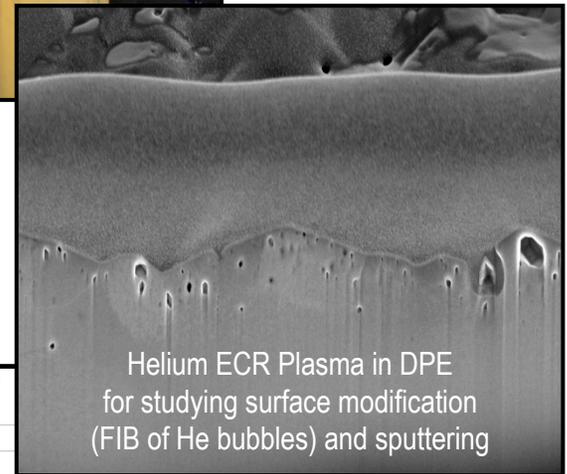
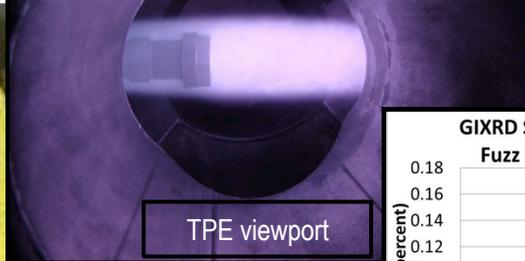
# SNL-CA PSI Science Center Research (FY10-14)

Fundamental surface science of H effects in ARIES



Tritium Plasma Experiment at INL

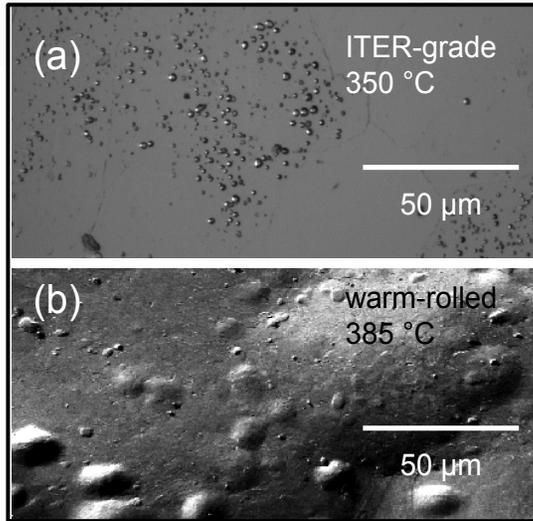
Inconel target  
He cooled  
 $T=1000\text{ }^\circ\text{C}$



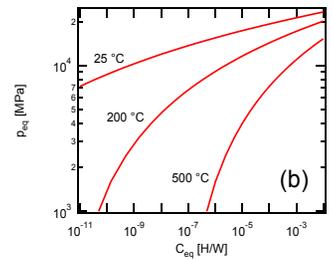
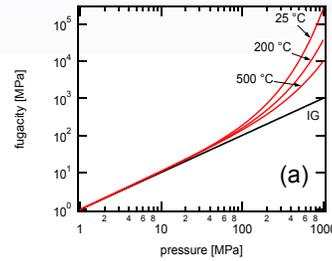
**The PSI SC efforts are heavily leveraged with our base program funding**



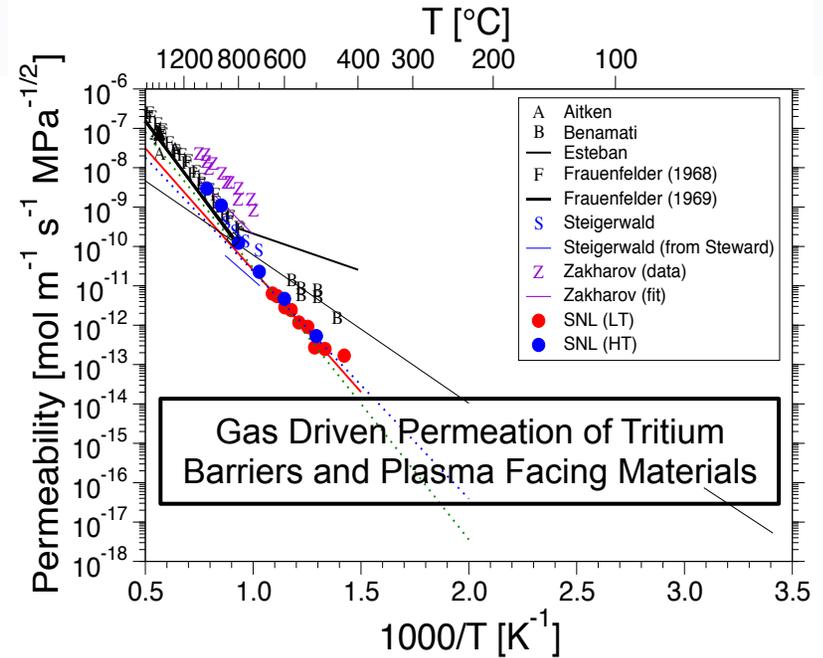
# SNL-CA base program (Technology & DIII-D)



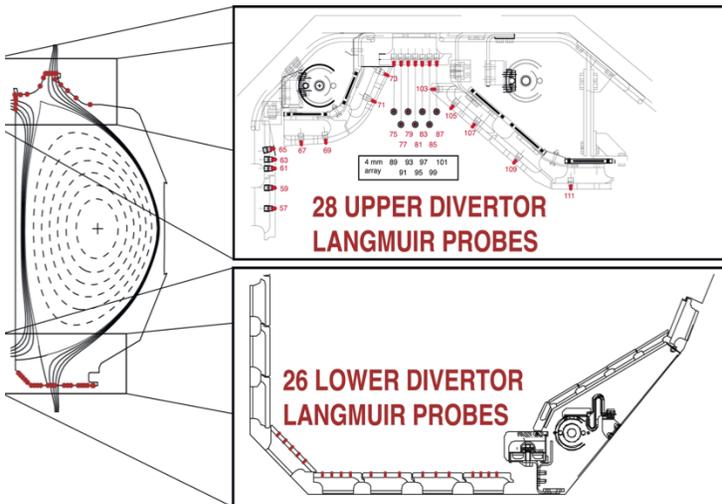
Hydrogen induced blistering of W



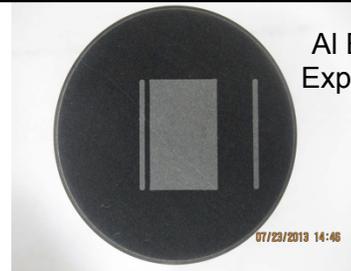
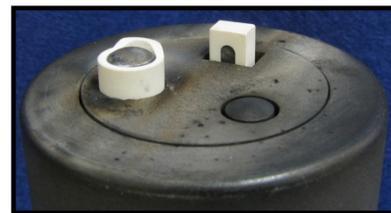
Equilibrium Pressure From Bubble Model



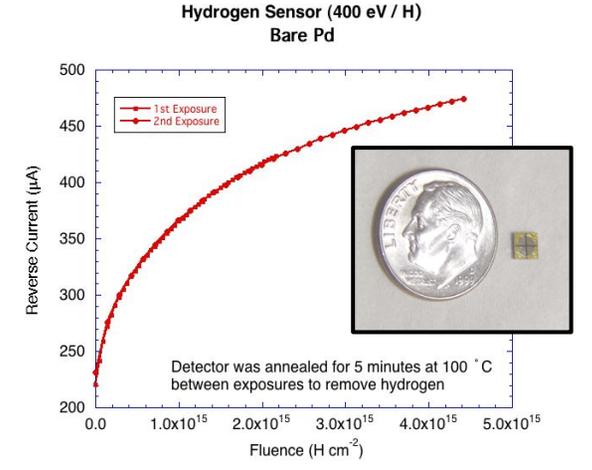
Gas Driven Permeation of Tritium Barriers and Plasma Facing Materials



Edge Plasma Characterization in DIII-D



Plasma Measurements and Erosion/Redeposition Experiments on DiMES (DIII-D)

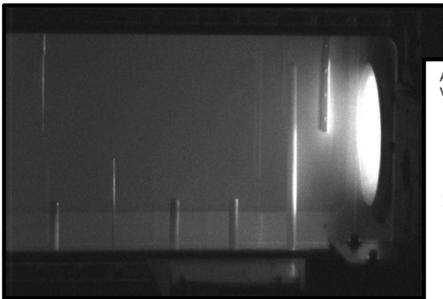


Hydrogen Microsensor Development

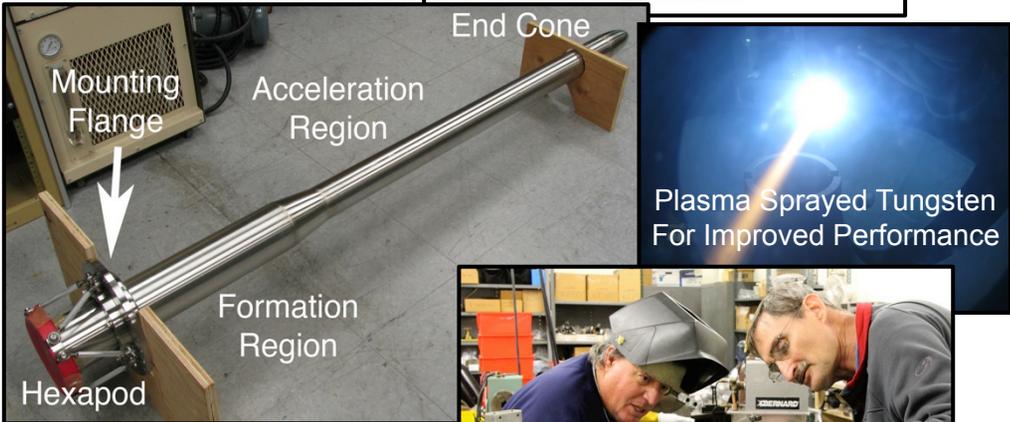
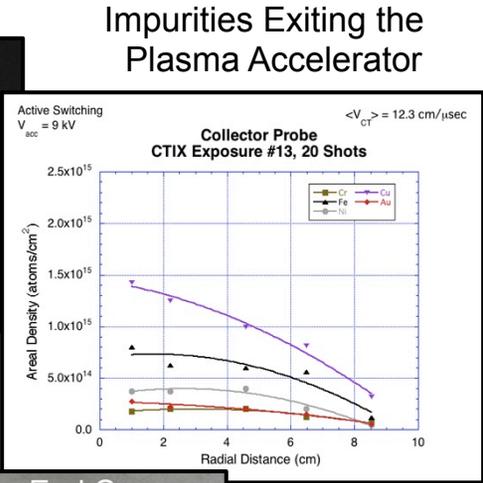
# Other fusion research at SNL-CA



Plasma Surface Interactions in the Compact Toroid Injection Experiment (started in FY10)



Collector Probes in CTIX



Improved Electrode Design



Final weld alignment

## Membrane Holder Photos

Spring loaded screw drive

Development of Plasma driven Permeation holder for TPE

Cooling fluid inlet

Helium carrier gas outlet

Helium carrier gas inlet

US-Japan Collaboration on PFC evaluation by Tritium Plasma, Heat, and Neutron Irradiation eXperiments

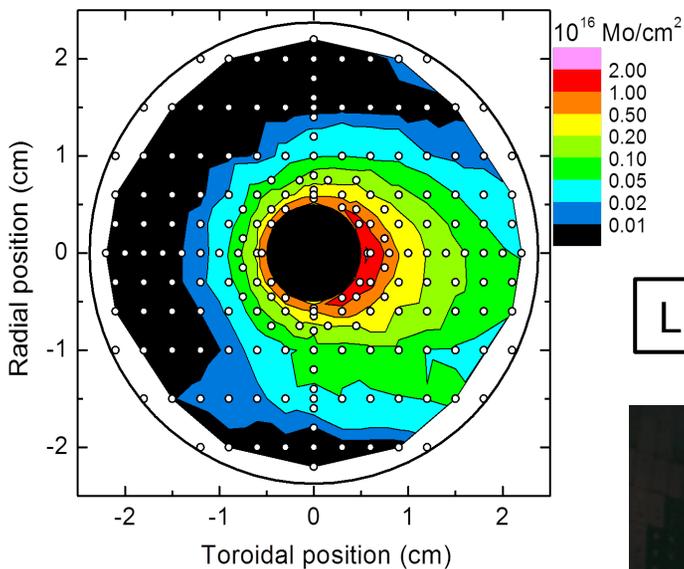
Spiral helium carrier gas channel

# Sandia Ion Beam Laboratory (NM)



DiMES experiments on DIII-D

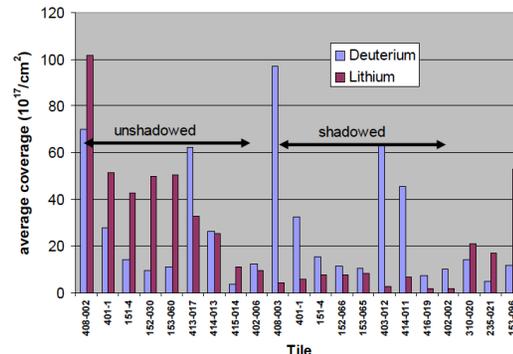
**Molybdenum erosion 0.5 nm/s**



19% redeposited locally with  $\lambda=2.3$  mm

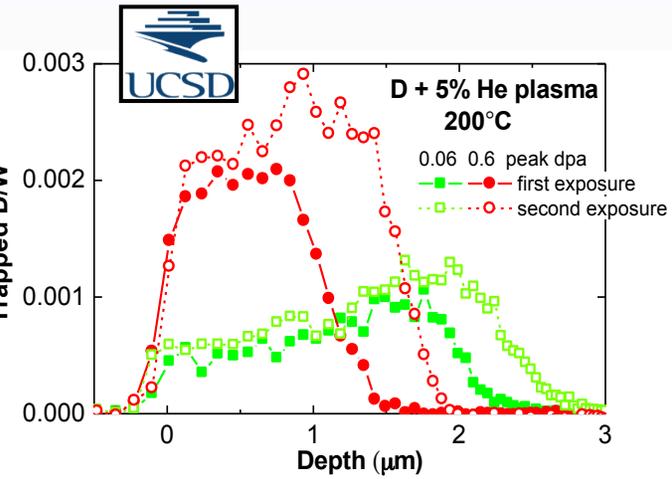


Average coverage of Li & D on each tile

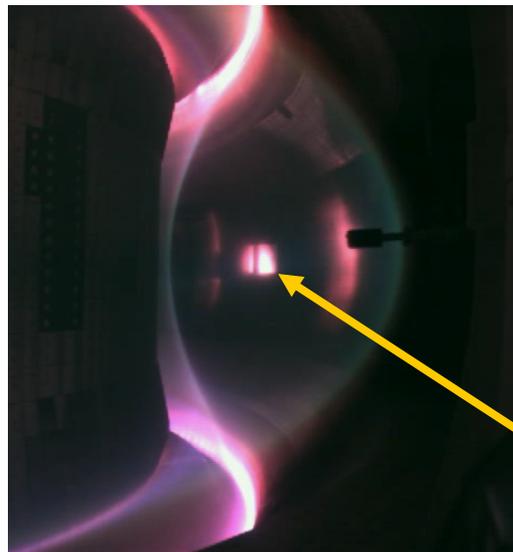


Tiles in Li shadow have ~ 10x less Li than unshaded tiles, whereas D coverage is similar.

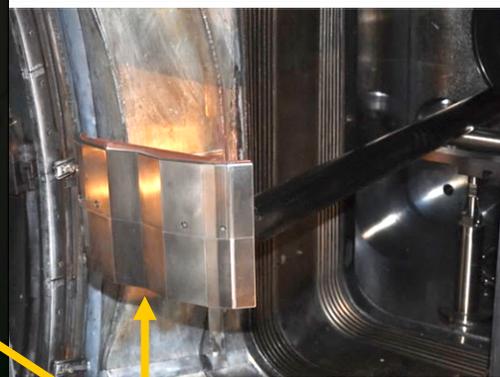
Li and D coverage on NSTX tiles



Deuterium trapping studies on damaged tungsten in PISCES



EAST/MAPES Material Erosion Experiment



MAPES



# US fusion program has a major focus studying plasma-material interactions (including erosion)

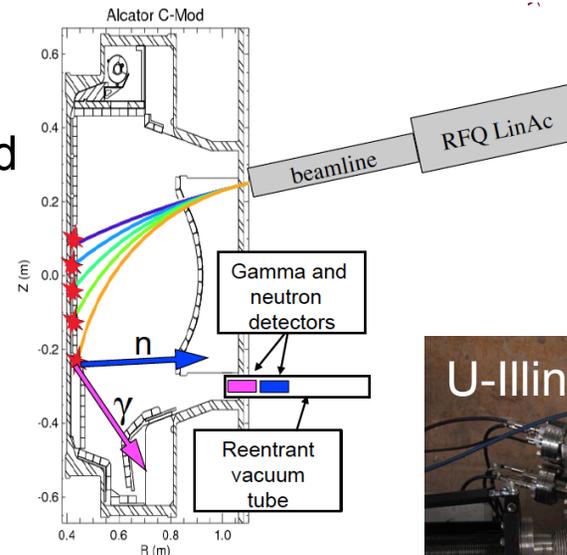
## ■ DIII-D

- DiMES (divertor) and MiMES (first wall) experiments provide erosion / redeposition data for plasma-material interaction model validation (REDEP/ WBC-ITMC code)



## ■ Alcator C-Mod

- High Z erosion and redeposition studied using a dedicated ion beam facility
- AIMS: Accelerator-based In-situ Materials Surveillance (compact RFQ LinAc injects 0.9 MeV D<sup>+</sup>)



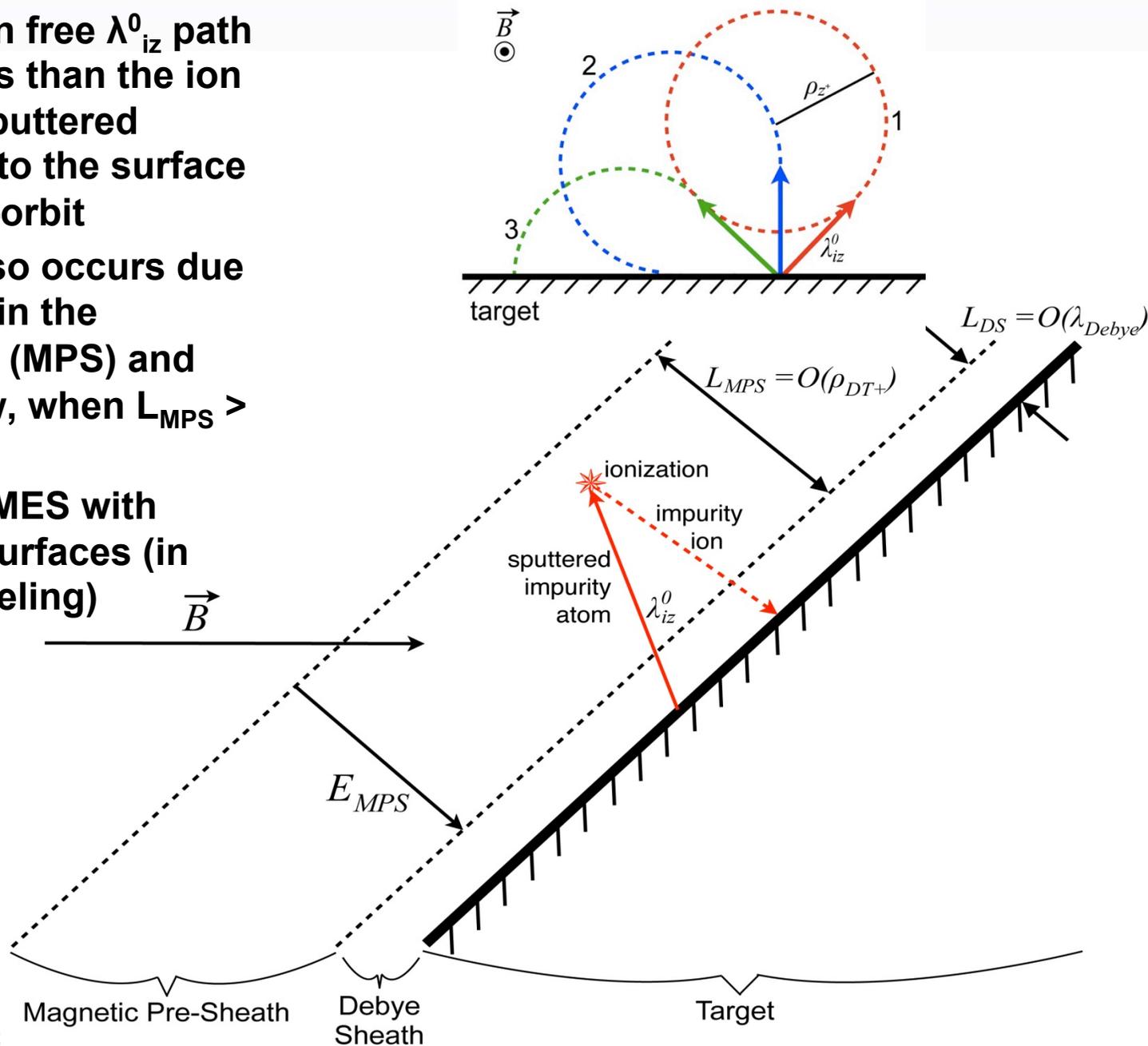
## ■ NSTX-U

- MAPP: Material Analysis and Particle Probe (sample exposure with in-situ TDS, XPS, LEISS, DRS)

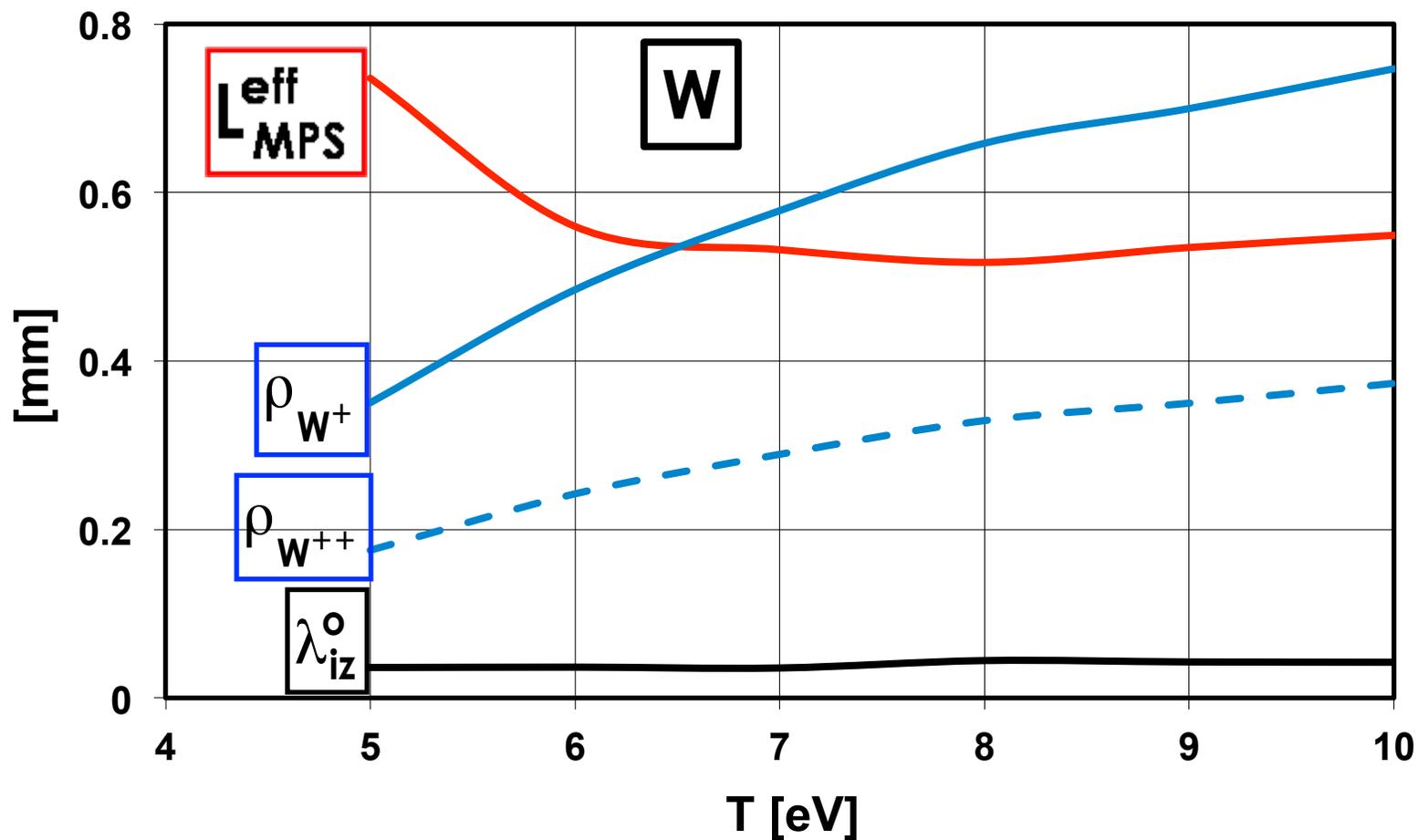


# Prompt redeposition by $\rho_{z+}$ / magnetic sheath of the divertor is expected to reduce net erosion

- If the ionization mean free  $\lambda_{iz}^0$  path is comparable or less than the ion Larmor radius  $\rho_{z+}$ , sputtered impurity ions return to the surface during the first gyro-orbit
- Fast redeposition also occurs due to the strong E-field in the magnetic pre-sheath (MPS) and friction with fast flow, when  $L_{MPS} > \lambda_{iz}^0$
- Demonstrated on DiMES with tungsten and moly surfaces (in agreement with modeling)



# For tungsten, prompt deposition should be effective in DT devices via both strong MPS forces and large $\rho_W$



ITER case with  $n_e = 10^{21} \text{ m}^{-3}$ ,  $B = 5 \text{ T}$

Courtesy of P. Stangeby

# Prompt deposition *is not* expected at the first wall

- **Much lower far scrape off layer (SOL) plasma density ( $\sim x10^{-3}$ ) will result in longer ionization mean free path  $\lambda_i^0$** 
  - Sputtered wall neutrals will penetrate into the far SOL and experience migration towards the divertor
- **The sputtering can be due to either plasma ions or charge exchange (c-x) neutrals**
- **While several efforts to characterize the first wall c-x flux occurred during the 80's / 90's, no current investigations exist on US tokamaks**
  - LENA (Low Energy Neutral Analyzer): PLT, ASDEX, Alcator C-Mod (installed only)
  - CRP (Carbon Resistance Probe): TMX-U, PLT, ASDEX, TFTR
  - Pd-MOS H-sensors (Palladium Metal-Oxide-Semiconductor)
    - ◆ Diode type: ZT-40M and TFTR
    - ◆ Capacitance type: DIII-D, NSTX
- **Is the flux of c-x neutrals important in FW erosion? What is the poloidal and toroidal distribution of the c-x flux? How will c-x induced erosion scale in future devices?**
- **Simple estimate on next viewgraph: physical sputtering of T using  $E=300$  eV (Eckstein 2002), normal incidence yield doubled to account for surface roughness, no sputtering by other plasma or wall species,  $P_{cx} = 0.05 P_{heat}$  (~Kukushkin for ITER)**

# Rough estimate of c-x induced FW erosion

For 300 eV T° c-x sputtering of walls	P <sub>heat</sub> [M W]	annual run time [s/year]	<b>beryllium</b> net wall erosion rate [kg/yr]	<b>boron</b> net wall erosion rate [kg/yr]	<b>carbon</b> net wall erosion rate [kg/yr]	<b>tungsten</b> net wall erosion rate [kg/yr]
<b>DIII-D</b>	<b>20</b>	<b>10<sup>4</sup></b>	<b>0.13</b>	<b>0.11</b>	<b>0.08</b>	<b>0.16</b>
<b>JT-60SA</b>	<b>34</b>	<b>10<sup>4</sup></b>	<b>0.22</b>	<b>0.19</b>	<b>0.15</b>	<b>0.27</b>
<b>EAST</b>	<b>24</b>	<b>10<sup>5</sup></b>	<b>1.6</b>	<b>1.2</b>	<b>0.82</b>	<b>1.8</b>
<b>ITER</b>	<b>100</b>	<b>10<sup>6</sup></b>	<b>77 [29*] {60***}</b>	<b>64</b>	<b>44 [53*] {54***}</b>	<b>92 [41*] {46***}</b>
<b>Vulcan</b>	<b>20</b>	<b>10<sup>7</sup></b>	<b>120</b>	<b>100</b>	<b>70</b>	<b>150</b>
<b>FDF</b>	<b>100</b>	<b>10<sup>7</sup></b>	<b>610</b>	<b>500</b>	<b>340</b>	<b>740</b>
<b>Reactor</b>	<b>400</b>	<b>2.5x10<sup>7</sup></b>	<b>6500</b>	<b>5300</b>	<b>3700</b>	<b>7900 [5000**]</b>

Courtesy of P. Stangeby

\* Kukushkin B2-EIRENE calculation

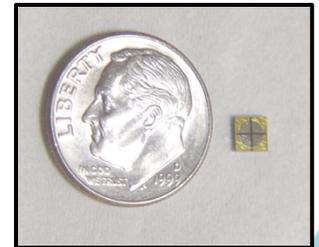
\*\*Lackner

\*\*\*Behrisch



# Techniques for measuring c-x flux

- **LENA (Low Energy Neutral Analyzer)**
  - Pros: provides good time and energy resolution
  - Cons: Available at only a few locations (usually one) due to size and complexity
- **CRP (Carbon Resistance Probe)**
  - Pros: compact and low power device, energy discrimination by array with varying overlayers, good time resolution
  - Cons: device saturates permanently at  $2 \times 10^{15}$  H/cm<sup>2</sup>
- **Pd-MOS H-sensors (Palladium Metal-Oxide-Semiconductor)**
  - Pros: compact and low power device, energy discrimination by array with varying overlayers, in-situ reset through heating of device
  - Cons: dosimetric (shot by shot), lifetime limited by charge trapping



Each device can play a role towards quantifying c-x flux in existing experiments

# Pd-MOS sensor use in confinement experiments

- First use on ZT-40M demonstrated dosimetric effect of sensors

Work by Bob Bastasz and Bob Hughes

R. Bastasz, J. Nucl. Mater. 162-164 (1989) 587.

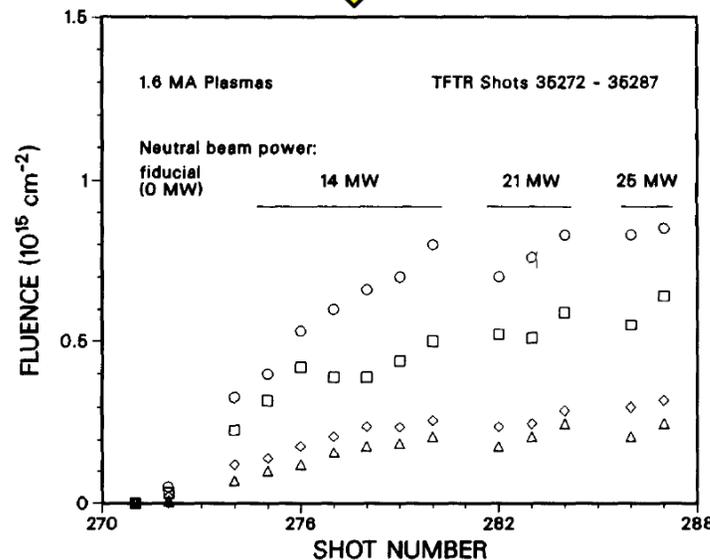
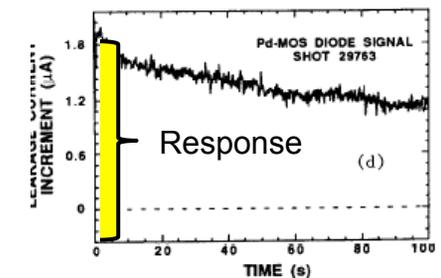
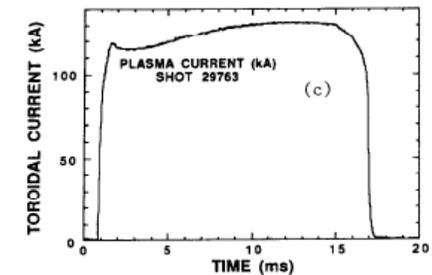
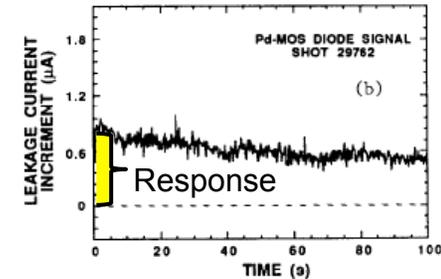
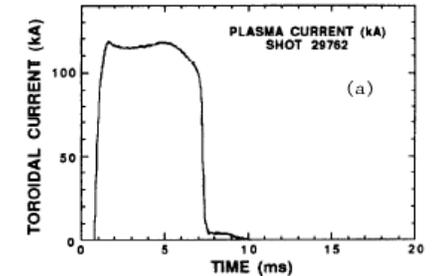
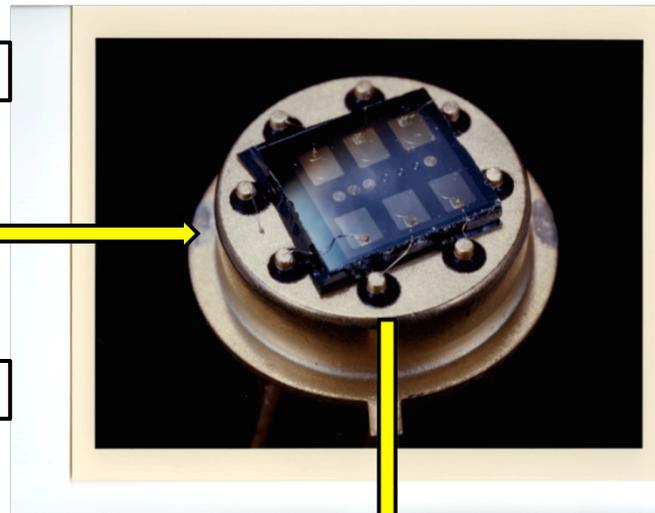
- An array sensor was used on TFTR to demonstrate energy resolving c-x measurement

R. Bastasz, J. Nucl. Mater. 176-177 (1990) 1038.

- Switched to capacitance type detectors for easy of fabrication

- Tests on DIII-D and NSTX were not successful

- Likely cause: charge production in thick oxide layer due to x-rays or high energy charged particles



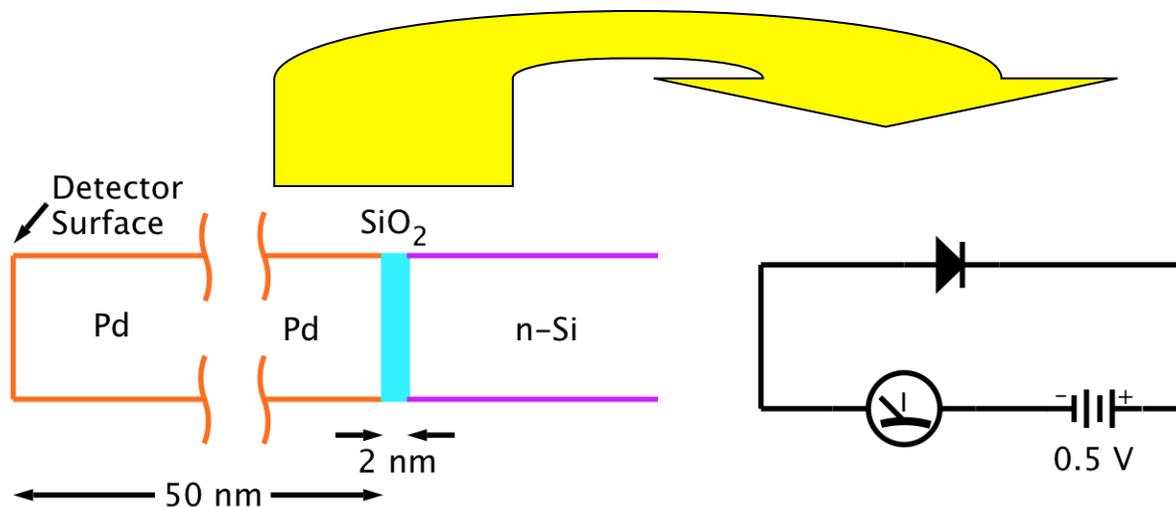
# Pd-MOS Schottky diode type H-sensors

- Diode type detectors will limit charge production due to thin tunnel oxide while a thicker Pd coating can protect from high energy particles
- Hydrogen entering the palladium metal diffuses rapidly, filling surface sites at the Pd-tunnel oxide interface, and changing the barrier height of the diode
  - Simple model of current under reverse bias

$$I_r = AT^2 \exp[-\phi_b / kT]$$

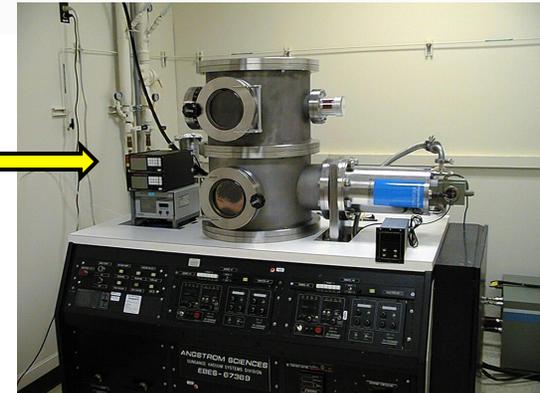
$$\phi_b = \phi_m - \chi$$

- A is a factor depending on the device size and applied voltage, T is the temperature, and  $\phi_b$  is the barrier height.  $\phi_m$  is the metal work function and  $\chi$  is the electron affinity of the silicon. Hydrogen at the interface changes  $\phi_m$  giving a large change in reversed current.

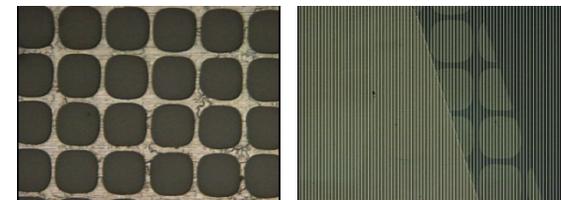
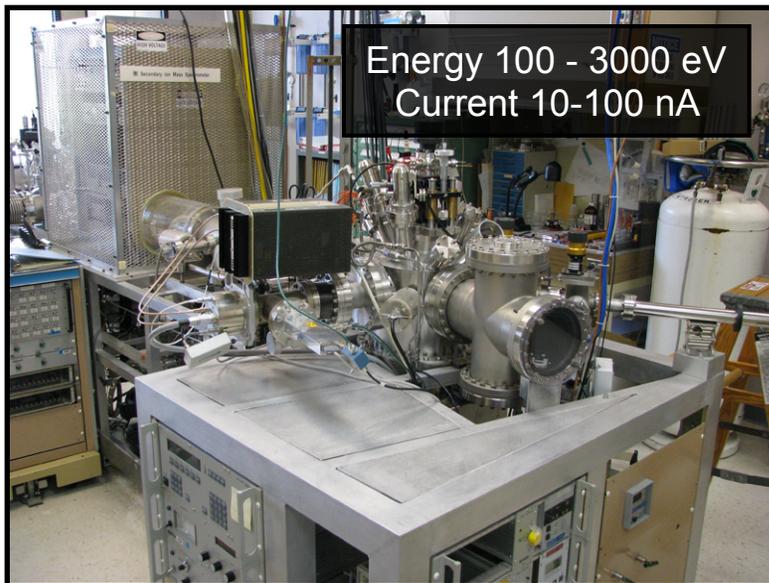


# Initial attempts to fabricate rugged Pd-MOS H-sensors

- **Wafers were stripped and 2 nm tunnel oxide grown in the MESAfab (silicon fab at SNL-NM)**
- **Pd films and titanium adhesion grids were deposited at SNL-CA (summer of 2010)**
  - Ti used to improve adhesion in the semiconductor industry
- **Each wafer produced many sensors (3.6 mm square)**
- **Sensors were evaluated using a mass and energy filtered ion beam (SNL-CA)**



10 keV dual e-beam evaporator used to deposit Pd, Ti, and Au under high vacuum ( $10^{-6}$  Torr). Typical deposition rates are  $50 \text{ \AA/s}$ .



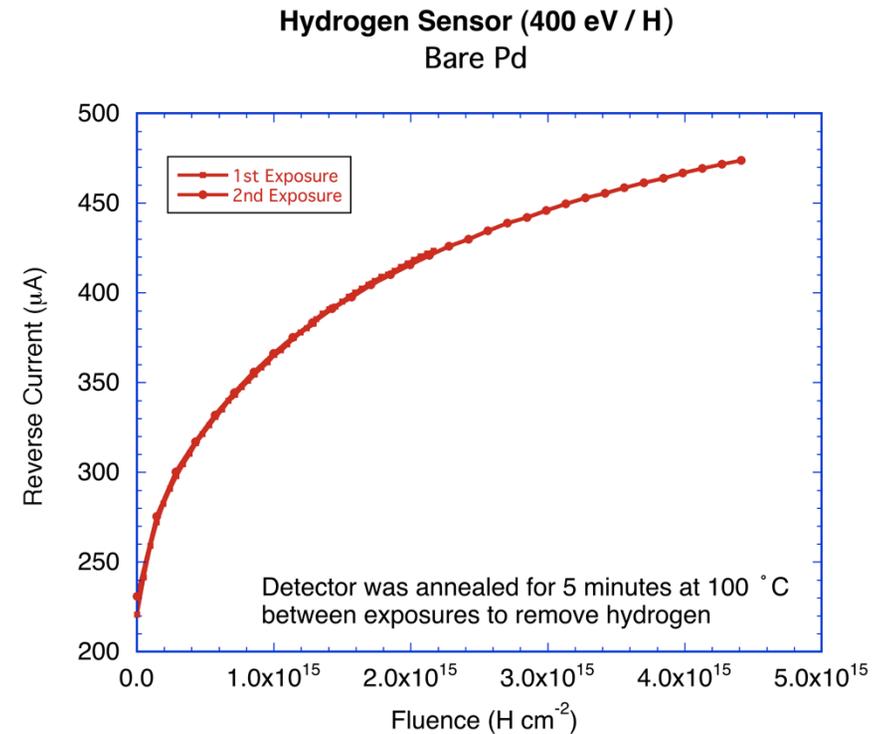
200  $\text{\AA}$  thick Ti posts at 200 dots/inch (48% active area) and 2000  $\text{\AA}$  thick Pd



200  $\text{\AA}$  thick posts at 200 dots/inch (76% active area) and 2000  $\text{\AA}$  thick Pd

# Ti posts unsuccessful, but diode-type H-sensors did exhibit similar behavior to previous detectors

- 20 nm Ti post array was too reactive and shorted the Pd metal-oxide interface
- Bare Pd sensors exhibited good response and reproducibility. Energy sensitivity needs further study.
- Exploring other options which will exploit extensive processing capabilities at the MESAfab
- Development of rugged detectors will require more than the 2010 several-month effort described here
- Leveraging of the MESAfab facility and expertise would require only a modest investment

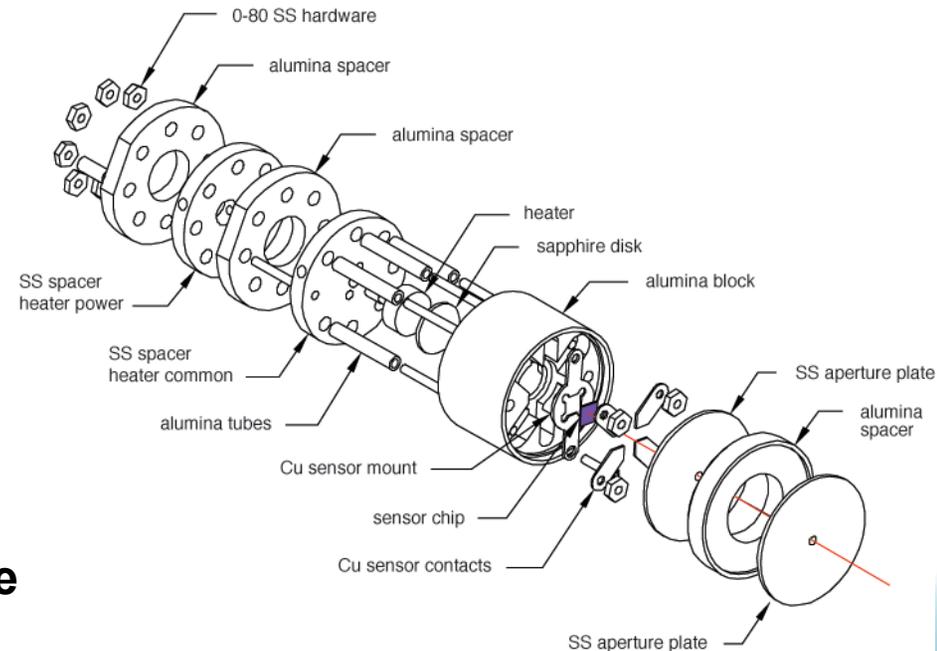


## MESAfab Key Fabrication Capabilities

- |  |  |
|--|--|
| <ul style="list-style-type: none"><li>• Photolithography Processes (Coat/Expose/Develop)</li><li>• Electron Beam (E-Beam) Lithography</li><li>• Reactive Ion Etch</li><li>• Wet Etch/Clean</li><li>• Oxidation and Diffusion</li><li>• Thin Films</li><li>• Chemical Mechanical Polishing (CMP) for planarization</li><li>• Ion Implantation</li><li>• Metrology</li><li>• Deep Reactive Ion Etch (DRIE)</li></ul> | <ul style="list-style-type: none"><li>• Electroplating</li><li>• Packaging</li><li>• MEMS Release</li><li>• Yield Learning</li><li>• Statistical Process Control</li><li>• III-V Compound Semiconductor Epitaxial Growth</li><li>• Mixed-Technology Integration and Processing</li><li>• 3D Integration</li><li>• Materials Characterization</li><li>• Failure Analysis</li><li>• Wafer Bonding and Thinning</li></ul> |
|--|--|

# Summary

- Unlike the divertor, the first wall of tokamaks will not experience prompt redeposition of sputtered material. Thus net erosion will be  $\sim$  gross erosion.
- Simple estimates and more complete simulations indicate that charge-exchange sputtering will result in large migration of first wall material in future devices.
- The development of Pd-MOS diode H-sensors would provide a tool that could quantify the poloidal and toroidal charge-exchange flux.
- Combined with the high energy and time resolution of a LENA, arrays of Pd-MOS devices could provide critical data for edge plasma – neutral model validation.



Pd-MOS H-sensor DiMES head

Continued interest at DIII-D and NSTX to evaluate Pd-MOS H-sensors